



# Simulazione di sistemi non lineari Nonlinear Current Allocator for Current Limit Avoidance at the JET Tokamak

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#### Outline



- Plasma Magnetic Modeling
- Plasma Shape Control Problem



#### Plasma Position and Shape Control at JET

- eXtreme Shape Controller
- Current Limit Avoidance System
- Experimental results





#### **Nuclear Fusion for Dummies**

## Main Aim

Production of energy by means of a fusion reaction

$$D+T \rightarrow {}^{4}\mathrm{He}+n$$



#### Plasma

- High temperature and pressure are needed
- Fully ionised gas  $\rightarrow$  Plasma
- Magnetic field is needed to confine the plasma





#### What is a Tokamak?



A tokamak is an electromagnetic machine containing a fully ionised gas (plasma) at about 100 million degrees within a torus shaped vacuum vessel. Poloidal and toroidal field coils, together with the plasma current, generate a spiralling magnetic field that confines the plasma.

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Simulazione di sistemi non lineari





### The JET tokamak - 1

- The Joint European Torus (JET) is an example of successful European collaboration.
- JET is still the world's largest tokamak
- JET has been built in the early eighties, and it was designed to allow the exploration of the plasma regimes in proximity of break-even, the condition at which the ratio between produced fusion power and input heating power is unity
- At the time of its construction, JET was a large step in scale from existing experiments





#### The JET tokamak - 2



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### Plasma axisymmetric model - 1

#### **Model Inputs**

The input variables are:

- The voltage applied to the active coils v
- The plasma current *I*<sub>p</sub>
- The poloidal beta  $\beta_p$
- The internal inductance *I<sub>i</sub>*

## $I_p, \beta_p$ and $I_i$

 $I_p$ ,  $\beta_p$  and  $I_i$  are used to specify the current density distribution inside the plasma region.





## Plasma axisymmetric model - 2

### **Model outputs**

Different model outputs can be chosen:

- fluxes and fields where the magnetic sensors are located
- currents in the active and passive circuits
- plasma radial and vertical position (1st and 2nd moment of the plasma current density)
- geometrical descriptors describing the plasma shape (gaps, x-point and strike points positions)







## Lumped parameters approximation

By using finite-elements methods, **nonlinear** lumped parameters approximation of the PDEs model is obtained

$$\frac{\mathrm{d}}{\mathrm{dt}} \Big[ \mathcal{M} \big( \mathbf{y}(t), \beta_{\mathcal{P}}(t), l_i(t) \big) \mathbf{I}(t) \Big] + \mathbf{R} \mathbf{I}(t) = \mathbf{U}(t) ,$$
$$\mathbf{y}(t) = \mathcal{V} \big( \mathbf{I}(t), \beta_{\mathcal{P}}(t), l_i(t) \big) .$$

where:

- y(t) are the output to be controlled
- I(t) = [I<sup>T</sup><sub>PF</sub>(t) I<sup>T</sup><sub>e</sub>(t) I<sub>p</sub>(t)]<sup>T</sup> is the currents vector, which includes the currents in the active coils I<sub>PF</sub>(t), the eddy currents in the passive structures I<sub>e</sub>(t), and the plasma current I<sub>p</sub>(t)
- $\mathbf{U}(t) = \begin{bmatrix} \mathbf{U}_{PF}^{T}(t) \ \mathbf{0}^{T} \ \mathbf{0} \end{bmatrix}^{T}$  is the input voltages vector
- $\mathcal{M}(\cdot)$  is the mutual inductance nonlinear function
- R is the resistance matrix
- $\mathcal{Y}(\cdot)$  is the output nonlinear function





### Plasma linearized model

Starting from the nonlinear lumped parameters model, the following plasma linearized state space model can be easily obtained:

$$\delta \dot{\mathbf{x}}(t) = \mathbf{A} \delta \mathbf{x}(t) + \mathbf{B} \delta \mathbf{u}(t) + \mathbf{E} \delta \dot{\mathbf{w}}(t), \tag{1}$$

$$\delta \mathbf{y}(t) = \mathbf{C} \, \delta \mathbf{I}_{PF}(t) + \mathbf{F} \delta \mathbf{w}(t), \tag{2}$$

where:

- A, B, E, C and F are the model matrices
- $\delta \mathbf{x}(t) = \left[\delta \mathbf{I}_{PF}^{T}(t) \, \delta \mathbf{I}_{e}^{T}(t) \, \delta I_{p}(t)\right]^{T}$  is the state space vector
- $\delta \mathbf{u}(t) = \left[ \delta \mathbf{U}_{PF}^{T}(t) \mathbf{0}^{T} \mathbf{0} \right]^{T}$  are the input voltages variations
- $\delta \mathbf{w}(t) = \left[\delta \beta_{p}(t) \ \delta I_{i}(t)\right]^{T}$  are the  $\beta_{p}$  and  $I_{i}$  variations
- $\delta \mathbf{y}(t)$  are the output variations

The model (1)–(2) relates the variations of the PF currents to the variations of the outputs around a given equilibrium





### **Plasma Shape Control**

- The problem of controlling the plasma shape is probably the most understood and mature of all the control problems in a tokamak
- The actuators are the Poloidal Field coils, that produce the magnetic field acting on the plasma
- The controlled variables are a finite number of geometrical descriptors chosen to describe the plasma shape





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#### Objectives

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#### Objectives

- Precise control of plasma boundary
- Counteract the effect of disturbances (β<sub>p</sub> and *l<sub>i</sub>* variations)
- Manage saturation of the actuators (currents in the PF coils)







### **Control scheme**



- The scenario is usually specified in terms of feed-forward currents  $I_{FF}(t)$ .
- It is convenient that the SC generates current references
- A PF currents controller must be designed





Plasma shape control at the JET tokamak

Two different shape controllers are available at the JET tokamak

- the *standard* Shape Controller (SC). This controller can be set in *full current control mode* (acting as a PF currents controler)
- the eXtreme Shape Controller (XSC)





XSC "philosophy"

- To control the plasma shape in JET, in principle 8 *knobs* are available, namely the currents in the PF circuits except *P*1 which is used only to control the plasma current
- As a matter of fact, these 8 knobs do not practically guarantee 8 degrees of freedom to change the plasma shape
- Indeed there are 2 or 3 current combinations that cause small effects on the shape (depending on the considered equilibrium).
- The design of the XSC is model-based. Different controller gains must be designed for each different plasma equilibrium, in order to achieve the desired performances





#### **XSC - Controller scheme**







## eXtreme Shape Controller (XSC)

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$$\delta \mathbf{g}(t) = \mathbf{C} \, \delta \mathbf{I}_{PF_N}(t).$$

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It follows that the plasma boundary descriptors have the same dynamic response of the PF currents.

• The XSC design has been based on the **C** matrix. Since the number of independent control variables is less than the number of outputs to regulate, it is not possible to track a generic set of references with zero steady-state error.

$$\delta \mathbf{I}_{PF_{N_{req}}} = \mathbf{C}^{\dagger} \delta \mathbf{g}_{error}$$





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- The determination of the controller gains is based on the Singular Value Decomposition (SVD) of the following weighted output matrix:

$$\widetilde{\mathbf{C}} = \widetilde{\mathbf{Q}} \, \mathbf{C} \, \widetilde{\mathbf{R}}^{-1} = \widetilde{\mathbf{U}} \, \widetilde{\mathbf{S}} \, \widetilde{\mathbf{V}}^{\mathsf{T}} \,,$$

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• The XSC minimizes the cost function

$$\widetilde{J}_{1} = \lim_{t \to +\infty} (\delta \mathbf{g}_{ref} - \delta \mathbf{g}(t))^{T} \widetilde{\mathbf{Q}}^{T} \widetilde{\mathbf{Q}}(\delta \mathbf{g}_{ref} - \delta \mathbf{g}(t)),$$

using  $\bar{n} < 8$  degrees of freedom, while the remaining  $8 - \bar{n}$  degrees of freedom are exploited to minimize

$$\widetilde{J}_{2} = \lim_{t \to +\infty} \delta \mathbf{I}_{PF_{N}}(t)^{T} \widetilde{\mathbf{R}}^{T} \widetilde{\mathbf{R}} \delta \mathbf{I}_{PF_{N}}(t) \,.$$





#### **XSC - Gap controller**







### **XSC and CLA**



- The XSC allows the SLs to directly specify the target shape, without specifying the PF current waveforms
- The PF current waveforms are *automatically* computed by the model-based control algorithm
- The PF currents may saturate during the experiment
- The Current Limit Avoidance System (CLA) has been recently designed and implemented to avoid current saturations in the PF coils when the XSC is used to control the plasma shape





#### The Current Limit Avoidance System - 1

- The CLA uses the redundancy of the PF coils system to automatically obtain almost the same plasma shape with a different combination of currents in the PF coils
- In the presence of disturbances (e.g., variations of the internal inductance *I<sub>i</sub>* and of the poloidal beta β<sub>p</sub>), it tries to avoid the current saturations by "relaxing" the plasma shape constraints
- Thanks to the CLA safe operations can be guaranteed





The Current Limit Avoidance System - 2



- The proposed current allocation scheme aims keeping the value of the plant inputs (PF currents) inside a desirable region, meanwhile ensuring a small tracking error on the plasma shape at steady state
- P\* is the plant steady-state gain





### The Current Limit Avoidance System - 3

The allocator equations are given by

$$\dot{x}_{a} = -KB_{0}^{T} \begin{bmatrix} I \\ P^{\star} \end{bmatrix}^{T} (\nabla J)^{T} \Big|_{(u,e)},$$
(3a)

$$\delta u = B_0 x_a, \tag{3b}$$

$$\delta y = P^* B_0 x_a \tag{3c}$$

- J(u<sup>\*</sup>, e<sup>\*</sup>) is a continuously differentiable cost function that penalizes (at steady-state)
  - large PF currents
  - large plasma shape error
- The key property of the current allocator algorithm (3) is that, for each constant current request of the XSC, it has a unique globally asymptotically stable equilibrium x<sub>a</sub><sup>\*</sup> coinciding with the unique global minimizer J(·,·)





#### The CLA scenario

When designing the current allocator, a large number of parameters must be specified by the user once the reference plasma equilibrium has been chosen:

- the two matrices P\* and B<sub>0</sub>, which are strictly related to the linearized plasma model
- the K matrix
- the gradient of the cost function *J* must be specified by the user. In particular, the gradient of *J* on each *channel* is assumed to be piecewise linear



**Figure:** Piecewise linear function used to specify the gradient of the cost function *J* for each *allocated* channel. For each channel 7 parameters must be specified.





#### The CLA Architecture



The CLA block is inserted between the XSC and the Shape Controller set in *Current Control Mode* 





#### The CLA block diagram







### **Experimental results**

The following strategy has been adopted to carry out the experiment

- first the reference pulse was run (pulse 81710), where the XSC without CLA has successfully controlled the plasma shape between 20 s and 23 s.
- The CLA has been then enabled starting from 21 *s*, in order to limit the currents in the four divertor coils within a range smaller than the available one
  - pulse 81712 both the currents in D2 and D3 have been limited between [-31.5, -10] kA and [-11, -2] kA
  - in pulse 81715 two further limits have been added, one on D1 ([-16.5, -4] kA) and one on D4 ([0, 6] kA)





#### Pulse 81712 - 1







#### Pulse 81712 - 2







Pulse 81715 - 1







#### Pulse 81715 - 2







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